



Jules Horowitz Reactor (JHR) status as of early 2025 following reassessment of the project and setting-up of « pre-JHR » phase before start-up of the reactor

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1. Why JHR, a new Material and Test Reactor in Europe?





An old fleet of Material Test Reactors in Europe

Country	Reactor	Power (MWth)	Age in 2024 (year)
Czech Republic	LVR15	10	68
Norway	HBWR	19	Shutdown mid 2018
Netherlands	HFR	45	64
Belgium	BR2	100	64
Poland	MARIA	30	48
France	OSIRIS	70	Shutdown end 2015
France	RJH	100	Under construction



New: PALLAS Project focused on radio-isotopes production





A research reactor supporting of the nuclear industry

An intense source of neutrons to accelerate the ageing of fuels and materials



1 year testing in JHR





8 to 10 years of operation in a nuclear power plant

The goal is to study and qualify the effects of irradiation on materials and nuclear fuels







JHR, in support to radioisotope production chain

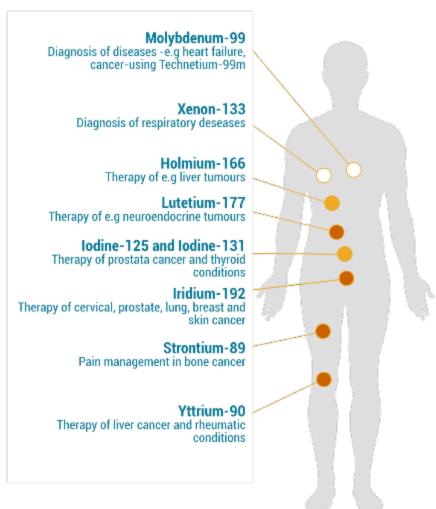
JHR is committed to irradiate between 25 and 50% of the European annual Molybdenum⁹⁹ targets

Molybdenum but not only...

JHR aims to become a leading player in securing the European supply of medical radioisotopes. Due to its high thermal neutron flux, it makes possible to produce a wide range of radioisotopes.

Thanks to:

- Large panel of irradiation locations in core or in reflector
- Irradiation devices and containers adapted to the target specificities
- Flow studies to ensure the arrival and departure of targets during the allocated time
- An active role and listening into the radisiotope community to anticipate and master the evolution of the needs
- Equilibrated business model



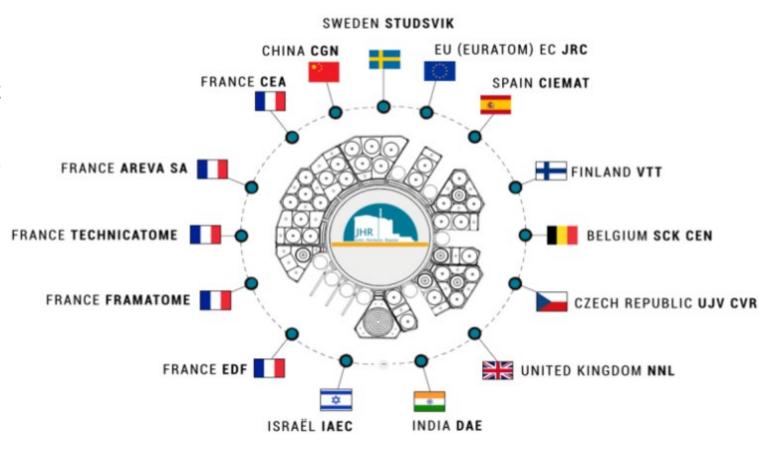






A Consortium of today 15 members for construction and operation

- CEA, owner, nuclear operator, project manager
- Each Consortium member contribute to finance the construction (in-kind or cash) and has in return access rights to the experimental capacity
- Partners are private and public entities
- CEA is mandated by the JHR Governing Board and the French Government to enlarge this international consortium







2. How does it work?



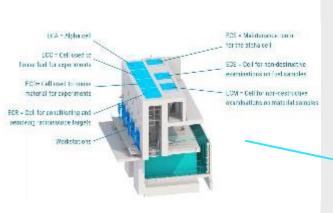


Organisation of the nuclear buildings



AUXILIARY BUILDING

Hot cells



Storage pools

Truck hall

Dimensions

BUR: Ф 36,6 m x H 43,1

m

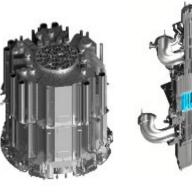
BUA: 50,3 x 46,4 m

H: 34,8 m





Reactor core





Water block
Continuous renewal

Underground Seismic pads





The core



OSIRIS reactor view



Highly instrumented devices

Online fission gas measurements

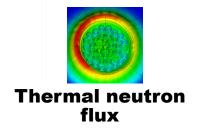
Up to 20 simultaneous experiments

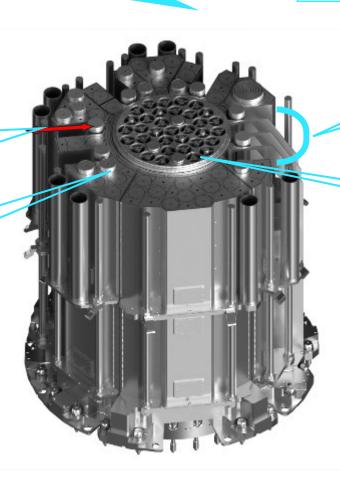
Displacement system

To adjust the neutron flux power needed

(power transient)

Some of radioisotope devices irradiation location





4 molybdenium⁹⁹ displacement systems

Material aging program





3] JHR status





Overall JHR Roadmap



Recovery Action Plan implementation (2019-2020),

Implementation of the 2021-2023 roadmap -> project review in 2023 (remaining activities and associated risks /project schedule up to commissioning and first experiments / cost at completion)

Nuclear Policy Council of 19 July 2023





- decision made to endorse the roadmap elaborated by CEA to finalize the installation of the reactor by 2032-2034
- key-role of the JHR for the extension of the existing nuclear fleet and for the deployment of nuclear reactors, both EPR and SMR recognized

As of mid-2023, full speed installation

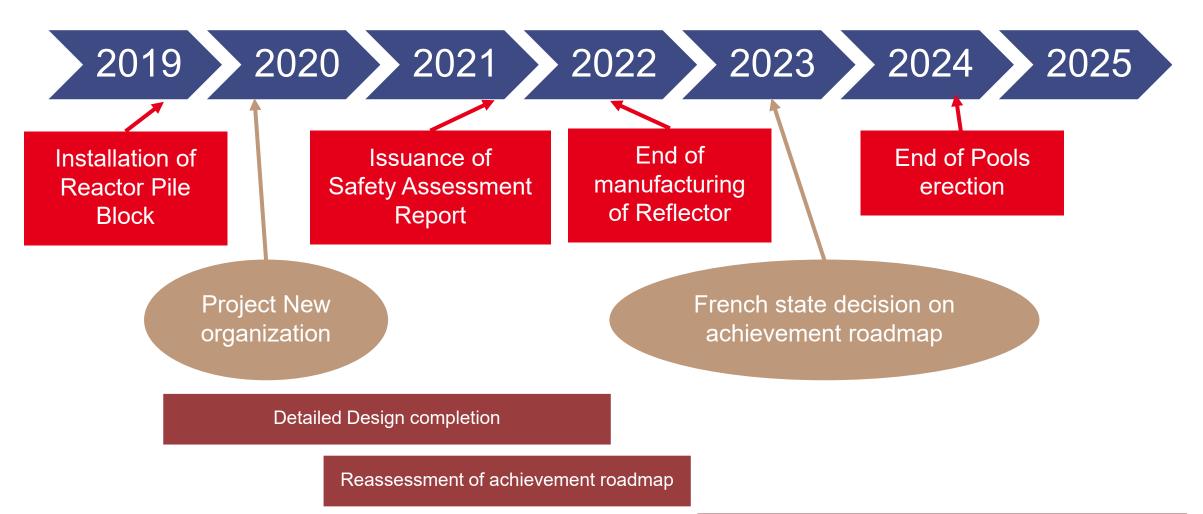
- Development of a reference schedule with the 3 main market holders
- Optimization of the schedule based on an action plan based on:
 - Integration of subsequent modifications in parallel with assembly and testing
 - Adjusting project resources to needs
 - Setting up acceleration measures
 - Risk anticipation
- Optimized planning is the common target of all project stakeholders: divergence at the end of 2032
- Aiming for the optimized schedule makes it possible to secure a start between the end of 2032 and the date obtained in the reference schedule (mid 2034)





Major achievements in the previous years





Electromechanical installation ramp-up up to full speed



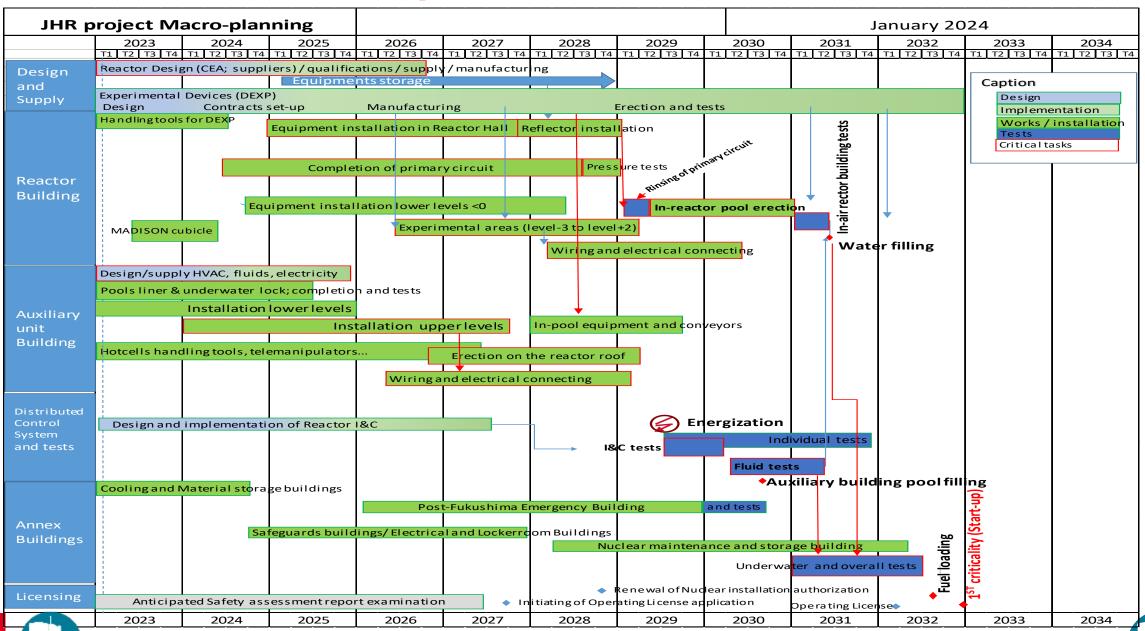
Major milestones of JHR project in the coming years

- 2025 : End of the detailed preliminary design phase of the 1st fleet of experimental devices
- 2025 : Ramp-up of the electromechanical assembly in the Reactor Building
- 2026 : Start of experimental area assembly
- 2027: End of assembly in the Auxiliary Nuclear Building
- 2028: End of assembly of the primary circuit
- 2029 : Commissioning of electrical distribution and start of functional tests
- 2030 : Filling of Auxiliary Nuclear Building pools
- 2031 : Filling of Reactor Building pools
- 2032 : Fuel loading and First Criticality (start-up)



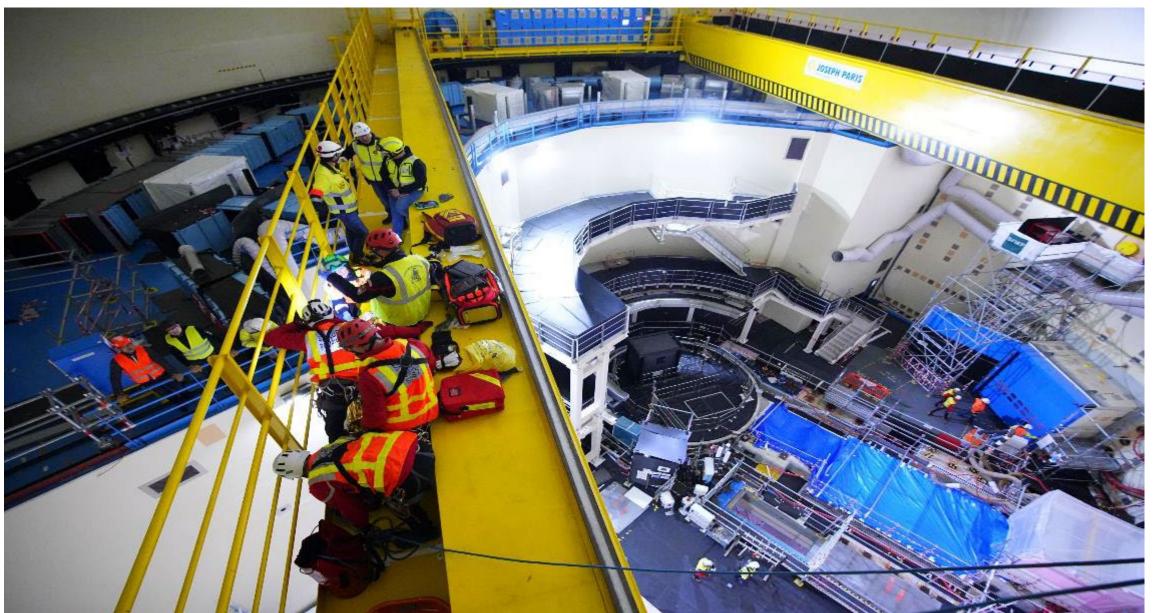


JHR completion schedule



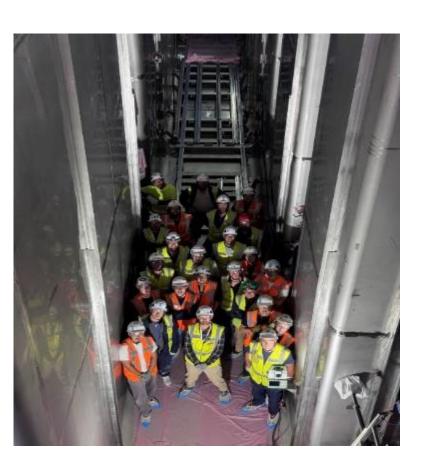
Reactor Hall





Recent Achievements (2024-2025)





Finalisation of pool liners in the Nuclear Auxiliary Building



Polar Crane re-qualification in the Nuclear Reactor Building



Test of the UGXR bench in the pool of TOTEM installation





Recent Achievements (2024-2025)





Concrete pouring in Safeguards building



Implementation of equipments Level -3

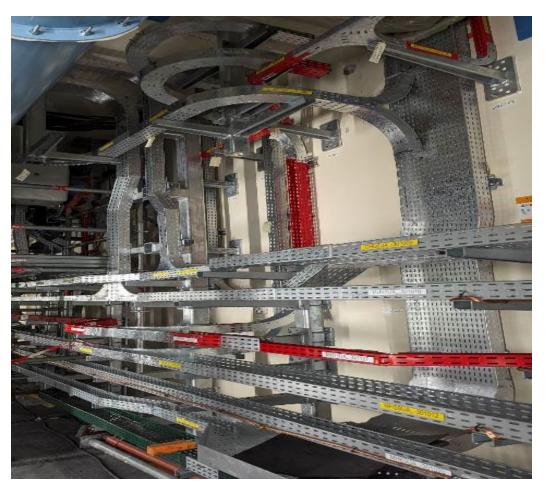


Recent Achievements (2024)









implementation of cable trays for electricity

Recent Achievements (2024-2025)







Installation of supports and pipes in the cooling circuit building



4] What kind of experiments and devices?





JHR hosting capabilities



In reflector

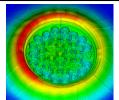
Up to 3.5E14 n/cm².s (th)

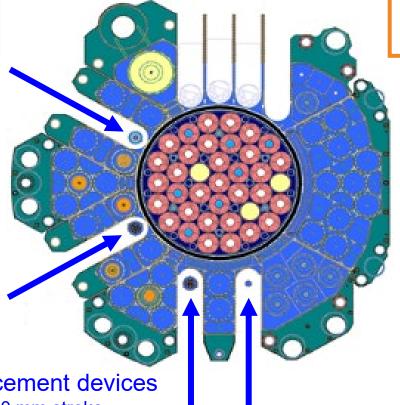
Fixed irradiation positions (Φ100 mm & Φ200 mm) and 4 displacement systems

LWR fuel experiments

Material ageing (low ageing rate Exple: RPV material)

Thermal neutron flux





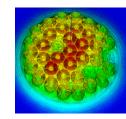
In core

Up to 5.5E14 n/cm².s (E> 1 MeV) Up to 1.E15 n/cm².s (E> 0.1 MeV)

7 small locations ($\Phi \sim 32$ mm) 3 large locations ($\Phi \sim 80$ mm)

Material ageing (high ageing rate) +
Gen IV fuel experiments

Fast neutron flux



Displacement devices
450 mm stroke
Vmax = 6 mm/s

→A large range of neutron fluxes and spectra

Enlargment of the first fleet of experimental devices'

Available for the JHR start-up

ADELINE

For LWR fuel testing under off-normal conditions (Power transients)

To be confirmed January 2026

CLOE

Corrosion loop for "Zr alloy Corrosion" and "Irradiation Assisted Stress Corrosion Cracking"

NEW

FUICA

Fuel capsule for LWR fuel selection and burn-up acquisition

MICA (x5)

For material testing under high dpa

Including pressurised samples with constant pressure loading **NEW**

OCCITANE

For Reactor Pressure Vessel steel testing (under low dpa)

MADISON

For LWR fuel testing under nominal conditions



Second/third fleets of experimental devices



Available few years after fleet 1

RISHI (IGCAR)

Loop for Material testing under Na environment

LORELEI

Fuel testing under accidental conditions (LOCA)

FUSERO A

Biaxial tests on ceramics (diagnostic windows)

MADISON 2 (second or third fleet)

For LWR fuel testing under nominal conditions

MICA - MELODIE

Specific sample holder for on-line monitoring of bi-axial deformation on a bi axially-loaded clad sample

LEBICA (CGN/CNPRI)

For material testing under Pb-Bi environment

CALIPSO

For material testing under high dpa and accurate temperature control

FUSERO C

Thermomechanical fatigue on divertor tiles (4 points bending or tension)



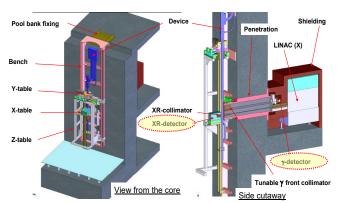
Non Destructive Examination Benches

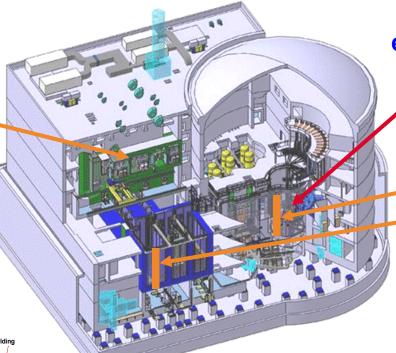


Gamma and X-Ray tomography systems

Multipurpose test benches

Coupled Gamma &X-ray bench





Test device examination in pools

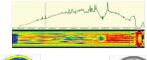
Neutron imaging system in reactor pool

Coupled X-ray & γ bench in reactor pool

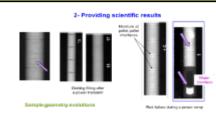
Coupled X-ray & γ bench in storage pool

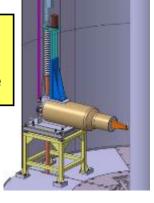
Neutron Imaging System

Initial checks of the experimental loading
Adjustment of the experimental protocol
On-site NDE tests after the irradiation phase









Preparation of JHR experiments using MTR's in operation through the OECD/NEA/FIDES2 Framework





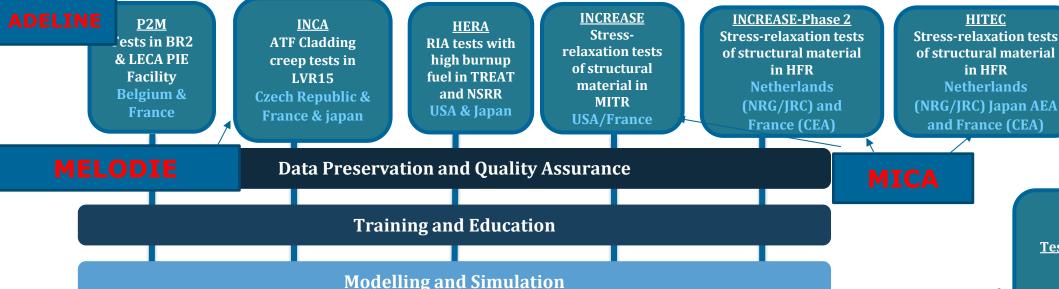
Framework Design 2nd Triennal Program of Work

Experimental programs complimented by cross-cutting pillars



Second <u>Framework for Irradiation Experiments</u> – FIDES-II

- NEA joint undertaking, established pursuant to Article 5 of the NEA Statues in co-ordination with the Nuclear Science Committee (NSC) and the Committee on the Safety of Nuclear Installations (CSNI)
- A stable, sustainable, reliable platform for fuel and material testing using nuclear research reactors (RRs) in NEA member countries
- · Generates experimental results and expertise for shared costs
- FIDES-II Program of Work includes 4 Joint Experimental Programmes (JEEPs) & 3 cross cutting pillars for the first triennial: 4 new JEEPs proposal under evaluation for 2nd and 3rd triennal



Instrumentation in support (CEA & INL)

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ATOMIC
Idaxation tests
ural material
n HFR
nerlands
C) Japan AEA
approx (CEA)

ATOMIC
Fuel capsule test in
ATR for innovative
fuel (TRISO, MOX
Gen2&3, MOX Gen4...)
USA/Japan/France

FUICA

26

LOCA-HBU

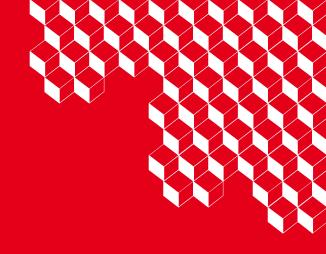
Test LOCA in TREAT
(INL)

<u>USA/Japan/France</u>

LORELEI

www.oecd-nea.org







jhrreactor.com/en JHR@cea.fr





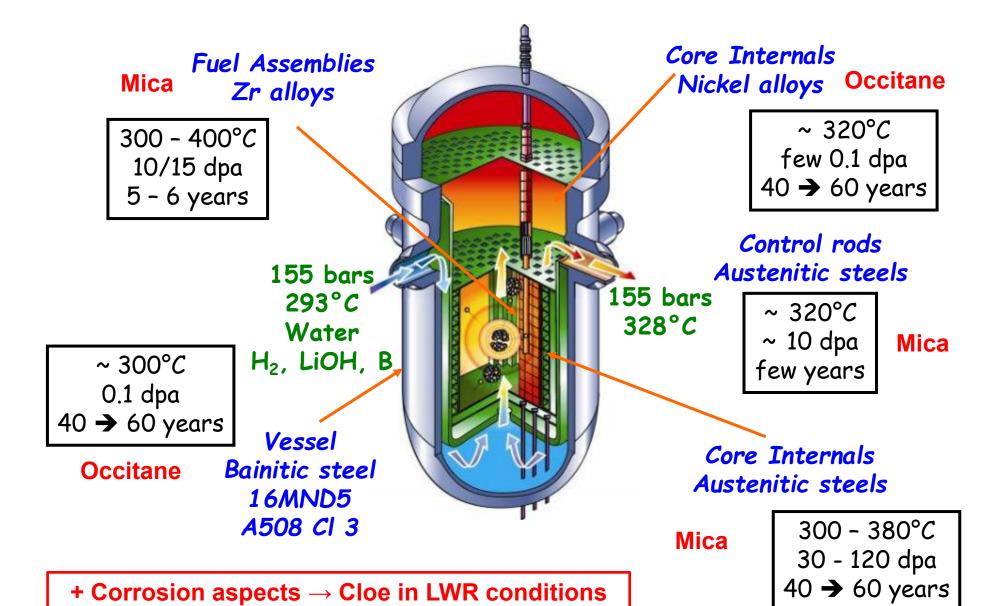
Appendices





Material experimental needs and JHR devices





cea

Fuel experimental needs and JHR devices



Selection / Characterization / Qualification / Safety tests

Burn-up acquisition

Fuel capsule, Madison,...

Madison with instrumented fuel rod, Adeline

Madison (nominal), Adeline (ramp tests)

- Main objectives
- ✓ Basis irradiation of several innovative products under similar conditions
- Main requirements
- ✓ High embarking capacity
- ✓ Few instrumentation
- ✓ Post irradiation examination

- Main objectives
- ✓ Measurement of physical properties under neutron flux
- ✓ Investigation of: Burn-up effect / Fission gas release / Pellet-Clad interaction / Chemical effect / Creep phenomena ...
- Main requirements
- ✓ High instrumentation
- ✓ Accurate control of environment conditions (steady or transient)
- ✓ Single effect experiments

Lorelei (LOCA)

- Main objectives
- ✓ Reproduction of environment conditions of power reactors in normal, incidental or **accidental** situations
- ✓ Envelope situations targeted
- Main requirements
- ✓ Good representativity of power reactor (steady and transient states)
- ✓ Long term or short term irradiations